

Ultra-trace element determination by neutron activation analysis in Acrylic material

Low Radioactivity Techniques 2017
Ewha Womans University, Seoul, Korea

Introduction

The **radio purity of materials is the essential condition** upon which rely the latest experiments of Rare Events Physics

The greatest risk

radioactive background
overlap the observable energy regions of
interest

Fundamental a selection of the
different components of the detector

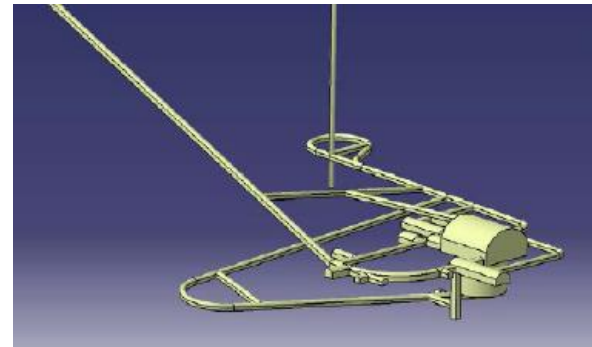
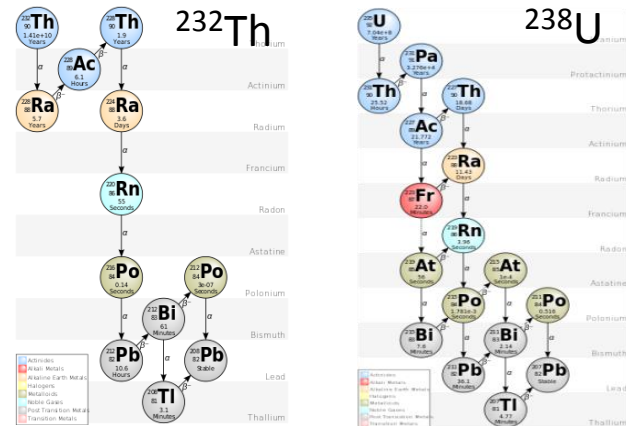


Typical requirements for contamination
level of detector materials

$$^{232}\text{Th} < 10^{-12} \text{ g/g (4 uBq/kg)}$$

$$^{238}\text{U} < 10^{-12} \text{ g/g (12 uBq/kg)}$$

$$^{40}\text{K} < 10^{-12} \text{ g/g (270 uBq/kg)}$$



Crucial to have
tools able to achieve that sensitivity

Acrylic material

Acrylic is a widely used material in many experiments in the physics of rare events

optical properties

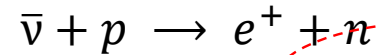
mechanical properties



Jiangmen Underground Neutrino Observatory



Inverse Beta Decay Reaction



Delayed signal ($\sim 200\mu\text{s}$) **2.2MeV γ -ray**

JUNO Central Detector

~ 20 kton LS detector



Acrylic Vessel

12cm thickness

Diameter: 35.5

It is crucial to carefully select acrylic material at \sim ppt level of contamination (^{238}U , ^{232}Th , ^{40}K) in order to reduce the accidental rate in the active volume of the detector

Acrylic samples

Samples received from China from
3 different companies

Each company has produces the
samples from sheets of 2 cm or 12
cm thickness

Company A

2 cm sheet production:
7 samples 248×248×20 mm³

Company B1

from 12 cm sheet production (panel
surface): 7 samples 248×248×20 mm³

Company B2

from 12 cm sheet production (panel
centre):
7 samples 248×248×20 mm³



Company C

from 12 cm sheet production (panel
centre):
7 samples 248×248×20 mm³

Available radioassay techniques @INFN-MIB



HPGe Spectroscopy*

Sensitivity on Th and U: ~ 10 mBq/kg

Bulk contaminations

γ emitting nuclides

Large amount of materials: few tens of kg

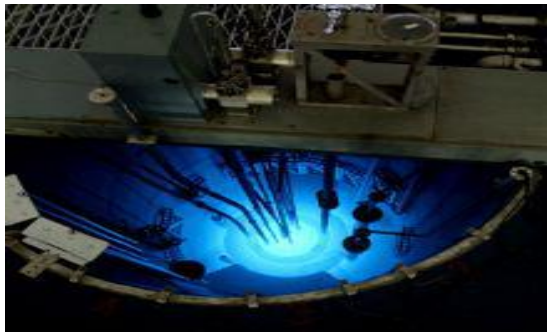


Alpha spectroscopy*

Sensitivity: 10^{-6} Bq/cm²

α emitting nuclides

Surface contaminations



Neutron Activation Analysis

Sensitivity on Th and U : \sim **uBq/kg**

Bulk contaminations

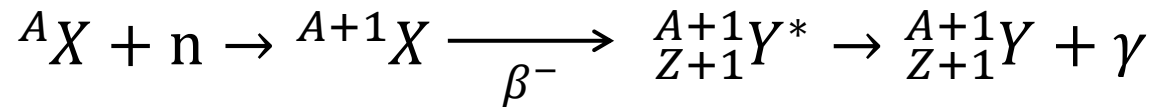
Primordial parents

Sample of few tens of g

*needs counting times from several weeks to months

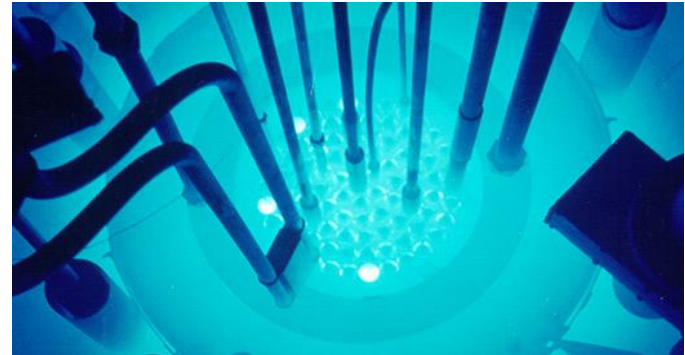
Neutron Activation Analysis (NAA)

The neutron activation process consists in the production of unstable isotopes through neutrons absorption by the nuclei present in the sample



The NAA technique consists of several steps:

Sample **exposure** to a neutron flux



→ **Extraction** of the irradiated sample and **measurement** of induced γ radioactivity (HPGe- detector)

→ **Calculation** of the quantity of precursor element (${}^A X$)

NAA key parameters:

Reaction cross section
Efficiency HPGe-detector
Neutron Flux
Irradiation Time
 γ emission and BR

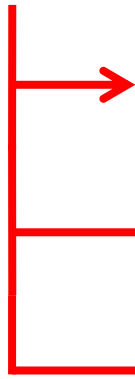
Neutron Activation Analysis

Comparative method

Sample and a reference Standard are prepared and irradiated



STD containing a known amount of the element to be measured



Time of irradiation is the same



$$T_{irr-Sample} = T_{irr-STD}$$

Same position in the irradiation channel



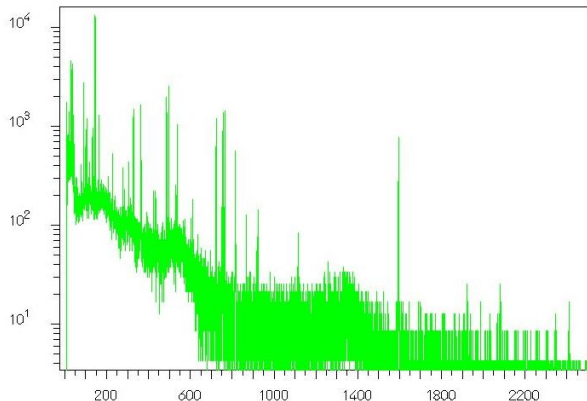
Neutron Flux

$$\phi_{Sample} = \phi_{STD}$$

Geometry_{Sample} = Geometry_{STD}
Geometry and size must be similar



Gamma spectroscopy - HPGe
Efficiency_{Sample} = Efficiency_{STD}



The activities of samples and standards are compared



Sample trace analysis

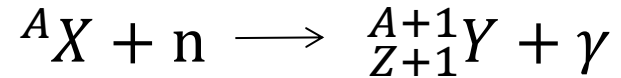
Neutron Activation Analysis

Three key ingredients:

high neutron flux

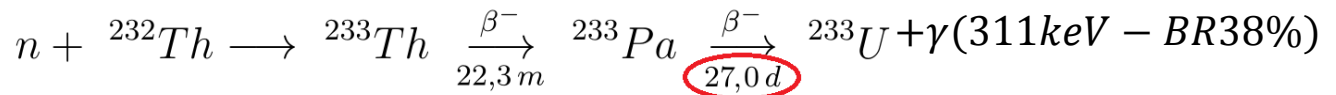
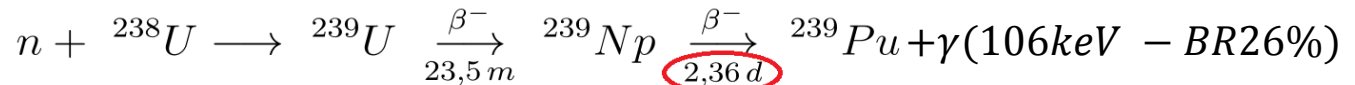
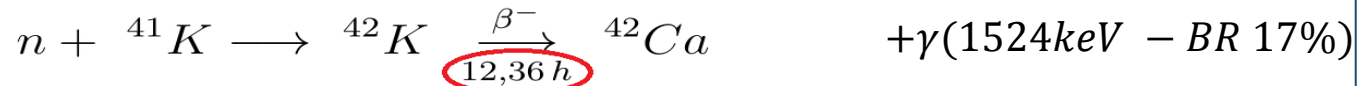
$$\phi \approx 10^{12} \div 10^{13} \text{ cm}^{-2} \text{ s}^{-1}$$

high enough neutron capture cross section



“convenient”

daughter nucleus
(γ emission, half-life
time)



Sensitivity depends on:

neutron exposure time

interferences in the matrix

background in the region
of the gamma emission



Care in the sample
preparation is
extremely important!

Advantage:

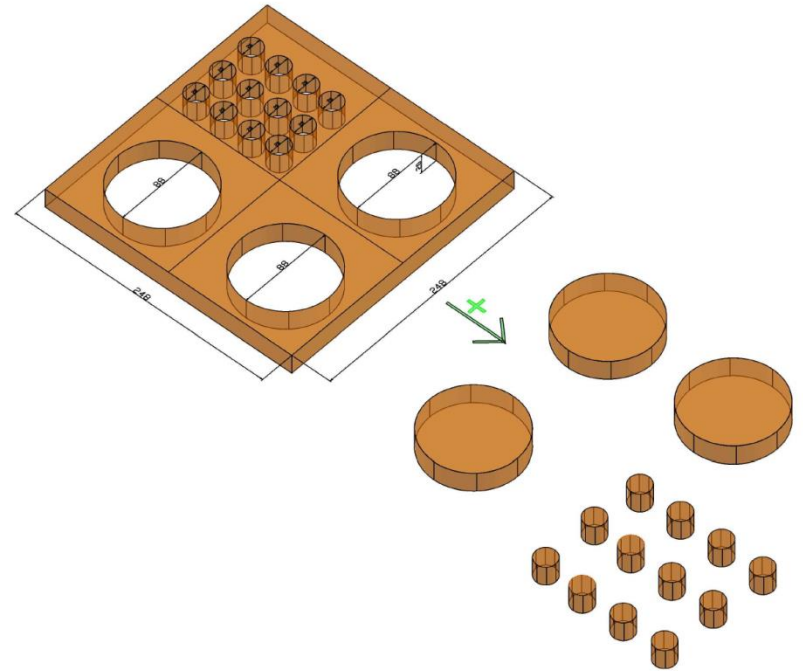
very high sensitivity :~ **1ppt**

Disadvantage:

sensitive only to radioactive chain progenitors

Sample preparation: Laser cut

To avoid the risk of contaminating the samples, we used the laser cutting technique

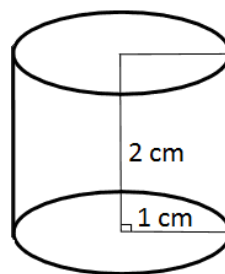


Extreme care was put to avoid contact of the samples with dirty surfaces

Sample preparation in clean room

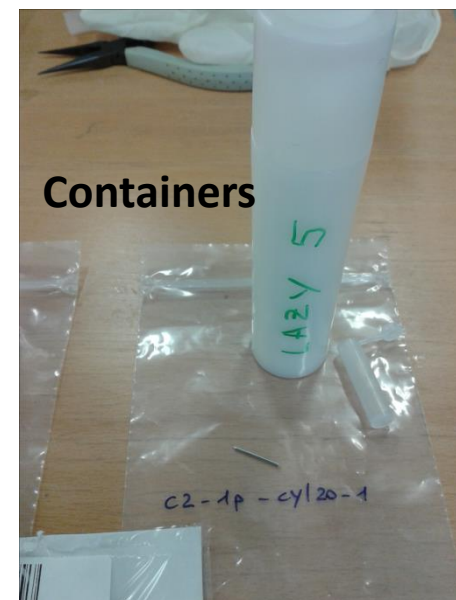
Acrylic samples washed several times in US bath (30°C) with MilliQ water in clean room 1000 atmosphere

Acrylic sample



Irradiation containers cleaned with ultra pure nitric acid solution (1%) in clean room 1000 atmosphere

Containers



STD preparation

Liquid Standard (1mg/ml)

3 Vials for each STD (0.02ml)

3 Vials blank with water

Liquid standards prepared
in separate containers from
samples

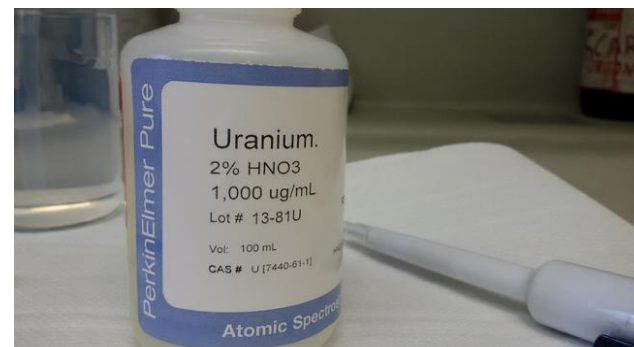
The vials were sealed and washed

Solid Standard:

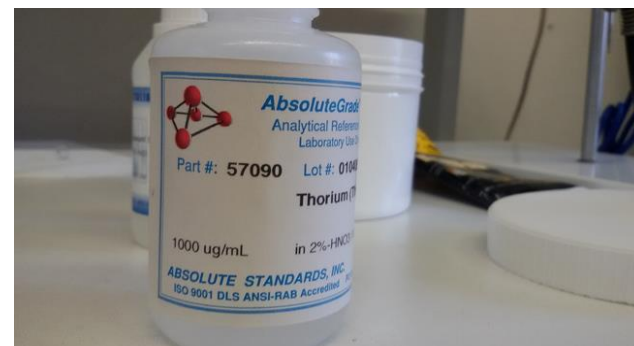
Al-Co (Co 0.5%wt) wire
standards put in every
irradiation channel as a cross-
check of the results



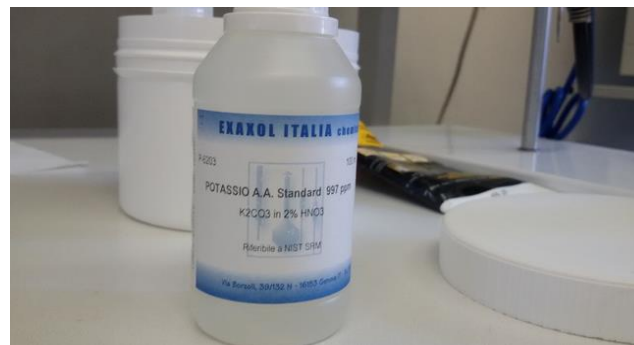
Uranium



Thorium

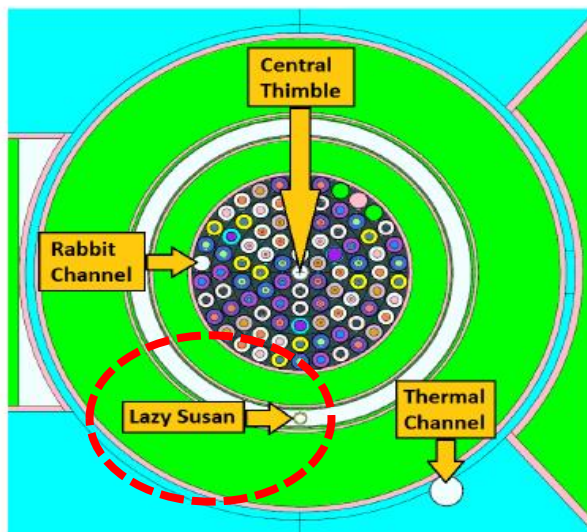


Potassium



NAA irradiation campaigns

TRIGA Mark II



Research reactor
(250 kW) - Pavia, Italy

LAZY SUSAN facility:

Flux of neutrons: $\approx 10^{12} \text{ cm}^{-2} \text{ s}^{-1}$

Irradiation Time: 12 hours

LAZY SUSAN facility is a rotary specimen rack in a circular well within the radial reflector

40 irradiation channels available around the reactor core, 7 adjacent channels were used

Irradiation Channel	LS1	LS2	LS3	LS4	LS5	LS6	LS7
Sample/STD liquid	A	STD2	B1	STD4	B2	STD6	C
STD solid	SS1	SS2	SS3	SS4	SS5	SS6	SS7

STD solid and samples were in the same channel

STD liquid and samples were in different channels to avoid contamination

Gamma spectroscopy



HPGe GePV

Beryllium Window

Relative efficiency: 25%

Coaxial geometry



Three detectors in Low background configuration

HPGe GeGEM

Coaxial geometry

Relative efficiency: 30%



HPGe GeKan

Coaxial geometry

Relative efficiency: 70%



HPGe BeGE

Planar geometry

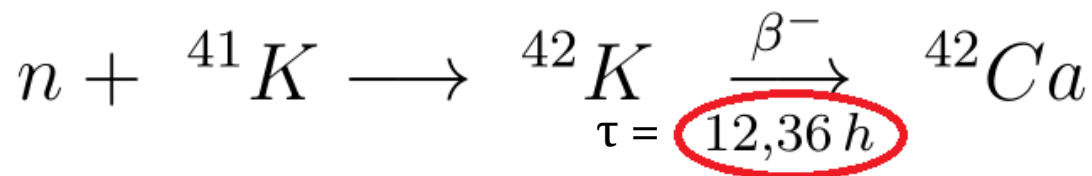
Relative efficiency: 50%

Carbon Window

Broad Energy detector



NAA results: ^{40}K



Isotopic Abundance K:

$\text{K}^{39} \rightarrow \sim 93\%$

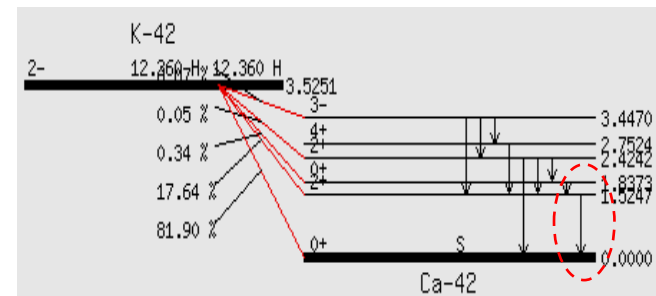
$\text{K}^{40} \rightarrow \sim 0,01\%$

$\text{K}^{41} \rightarrow \sim 7\%$

Untreated samples

Sample	$^{40}\text{K}(\text{ppt})$
A	0.7 ± 0.3
B1	0.7 ± 0.2
B2	0.7 ± 0.2
C	0.9 ± 0.2

limits @ 90% C.L.



$\gamma\text{-ray(keV)}$	BR(%)
312.6	0.34
1524.6	17.64

We checked the hypothesis that contamination is mainly located on surfaces of the samples.



We decide to remove a thin surface layer

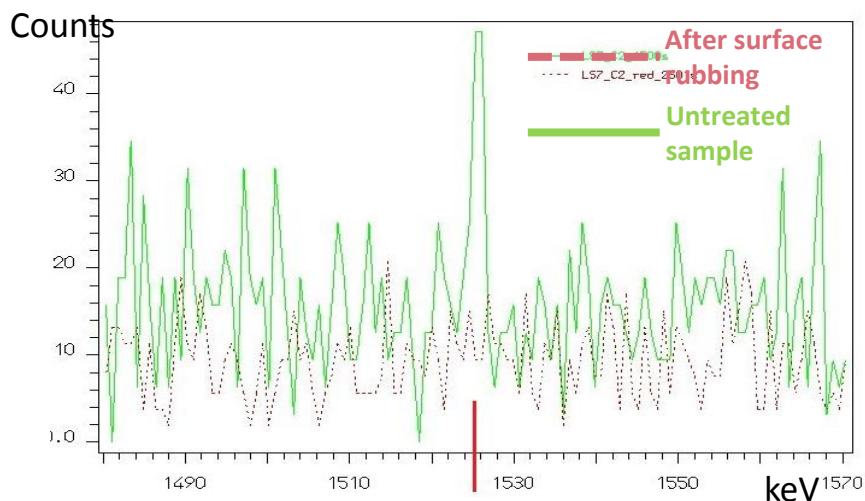
NAA results: ^{40}K

Problem of surface handling!

Sample	Mass(g)	Mass(g)
A	6.1786	5.754
B1	6.973	5.6672
B2	6.9494	6.1678
C	7.2181	6.441

Untreated samples After surface rubbing

Sample	$^{40}\text{K}(\text{ppt})$	$^{40}\text{K}(\text{ppt})$
A	0.7 ± 0.3	<0.1
B1	0.7 ± 0.2	<0.5
B2	0.7 ± 0.2	<0.4
C	0.9 ± 0.2	<0.1



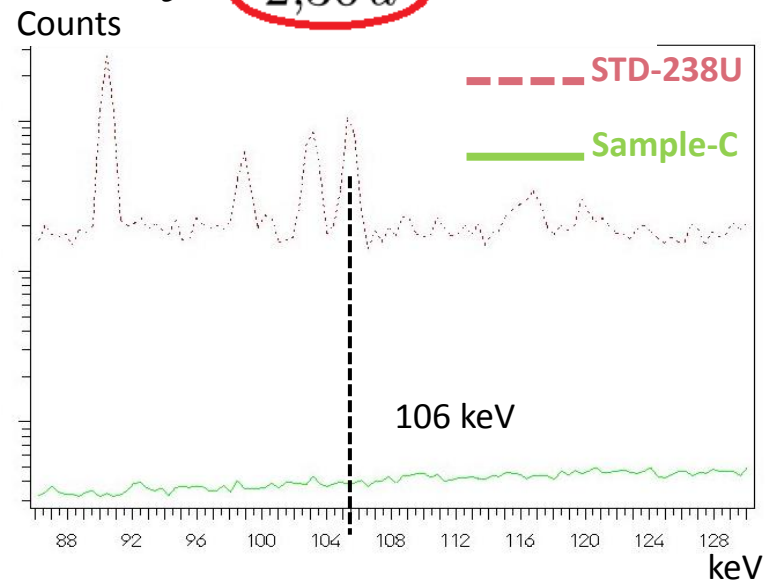
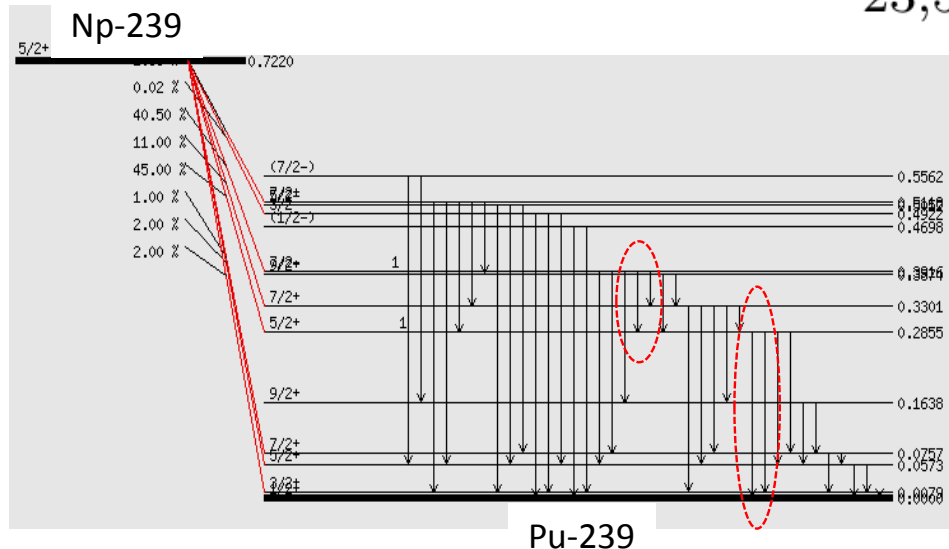
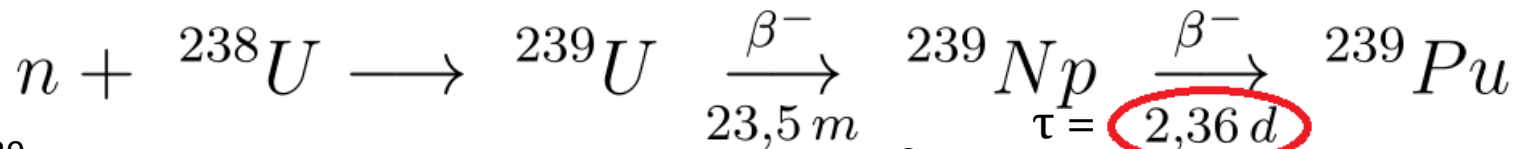
Sandpaper



The contamination was concentrated on the sample surface!

The reported limit on the presence of ^{40}K in acrylic are among the best ever achieved

NAA results: ^{238}U



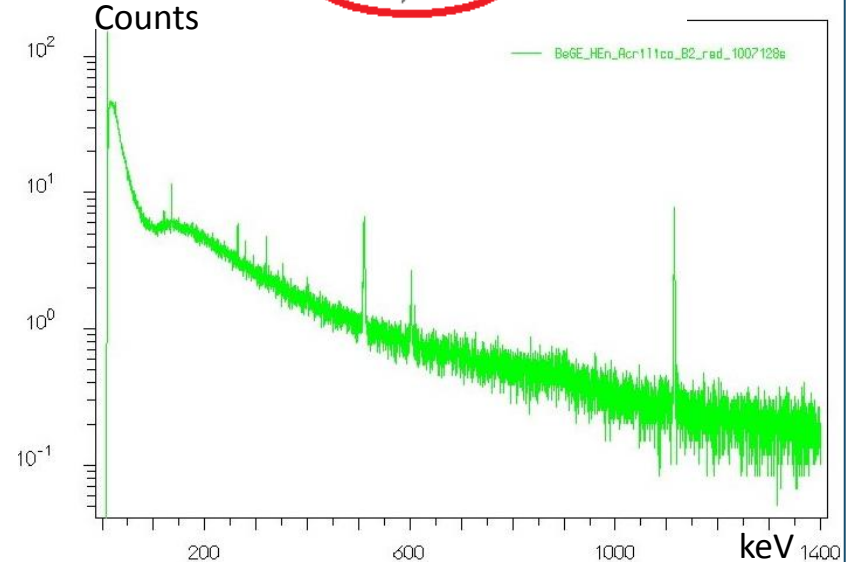
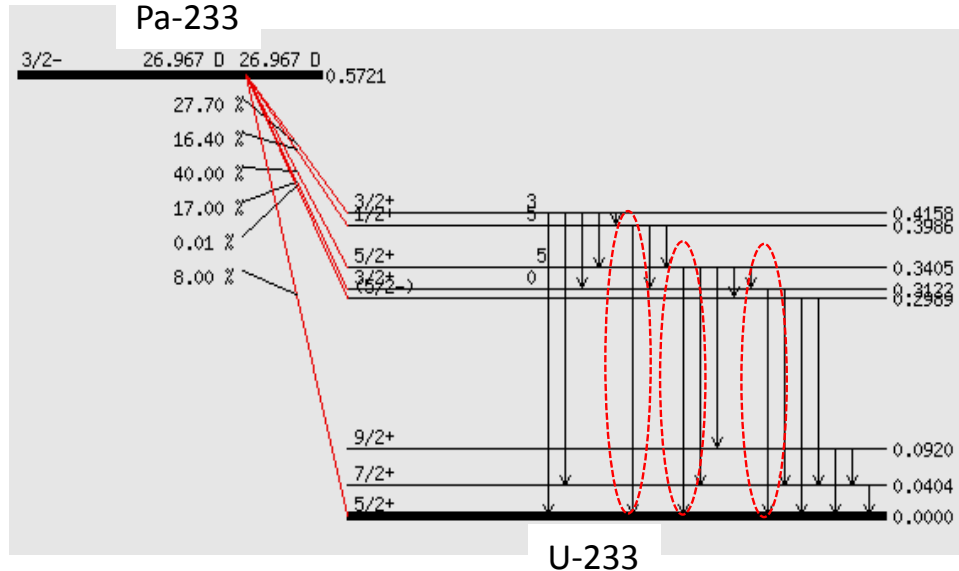
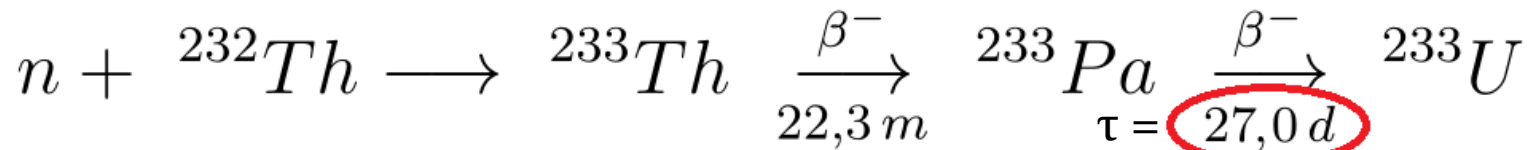
γ -ray(keV)	BR(%)
10612	26.3
228.18	11.14
277.60	14.44

Sample	^{238}U (ppt)
A	<2.0
B1	<3.2
B2	<2.0
C	<0.7

Measurements were carried out on 3 HPGe detectors

limits @ 90% C.L.

NAA results: ^{232}Th



γ -ray(keV)	BR(%)
300.13	6.63
311.90	38.5
340.48	4.45

Measurements were performed
~ 1 month after the irradiation

Sample	^{232}Th (ppt)
A	<2.9
B1	<2.4
B2	<2.5
C	<1.4

limits @ 90% C.L.

Conclusions

A methodology based on
neutron activation analysis (NNA) combined with treatment of the sample surfaces
has been developed to determine K, Th and U content in
Acrylic samples

By this methodology:

→ 10^{-12} g/g level has been achieved for ^{238}U and ^{232}Th

→ 10^{-13} g/g level has been achieved for ^{40}K

Backup slides

Summary

		GePV Untreated	GePV treated	GeKan	GeGEM
Sample	STD	⁴⁰ K(ppt)	⁴⁰ K(ppt)	⁴⁰ K(ppt)	⁴⁰ K(ppt)
A	2	0.7±0.3	<0.1	<0.1	
B1	2	0.7±0.2	<0.5		<0.2
B1	4	0.7±0.2	<0.5		<0.2
B2	4	0.7±0.2	<0.4		<0.4
B2	6	0.7±0.2	<0.4		<0.4
C	6	0.9±0.2	<0.1	<0.1	

		GeKan	GeGEM	BeGE
Sample	STD	²³⁸ U(ppt)	²³⁸ U(ppt)	²³⁸ U(ppt)
A	2	<2	<2	
B1	2	<7	<3	
B1	4	<8	<3	
B2	4		<2	
B2	6		<2	
C	6	<1		<0.7

		GeKan	GeGEM	BeGE
Sample	STD	²³² Th(ppt)	²³² Th(ppt)	²³² Th(ppt)
A	2	<3		<3
B1	2			<2
B1	4			<3
B2	4			<3
B2	6			<3
C	6		<3	<1

Measurements made with different standards are independent and are always compatible

STD solid

Channel	LS1	LS2	LS3	LS4	LS5	LS6	LS7
Sample/ STD liquid	A	STD2	B1	STD4	B2	STD6	C
STD solid	SS1	SS2	SS3	SS4	SS5	SS6	SS7

Al-Co wires



Integral flux

$$\Phi = \frac{R}{N\sigma_{eff}}$$

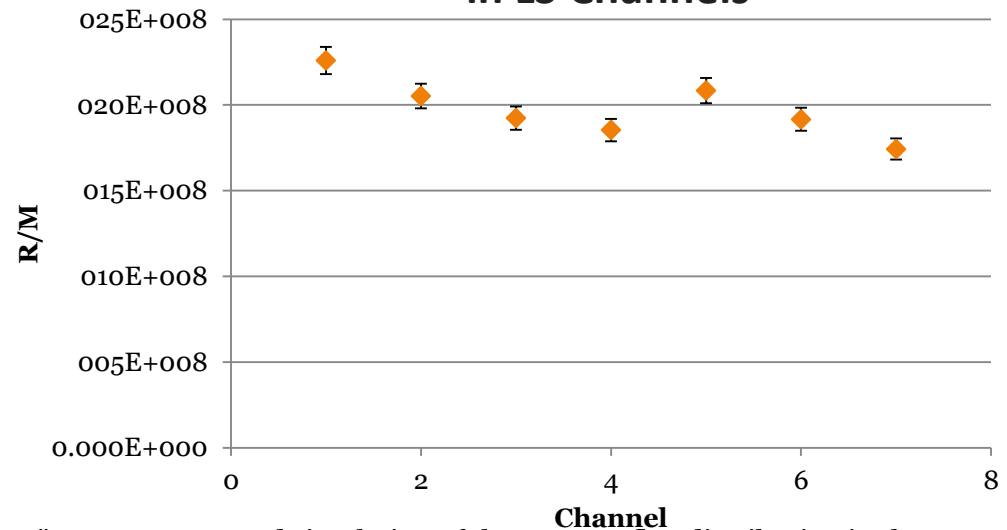
σ_{eff} : calculated
from MCNP

Activation Rate

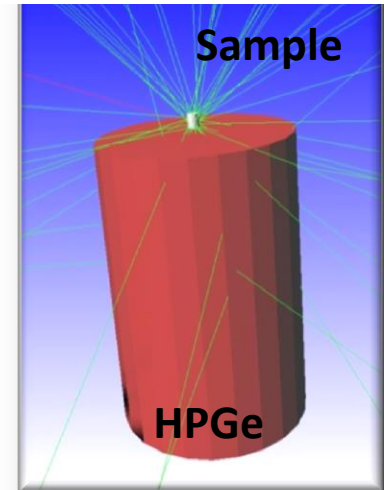
$$R \sim n_{dec} = \frac{C_{meas}}{C_{sim}} n_{sim}$$

Verification of the uniformity of the flux
in LS Channels

Channel	Mass(mg)
1	34.25
2	29.92
3	31.44
4	37.56
5	32.30
6	34.25
7	31.43



MC simulation



Reference: "Measurement and simulation of the neutron flux distribution in the TRIGA Mark II reactor core" Corresponding author: D.Chiesa

Efficiency: corrective factor

Acrylic Sample and STD liquid
have a different geometry

$$Efficiency_{Sample} \neq Efficiency_{STD}$$

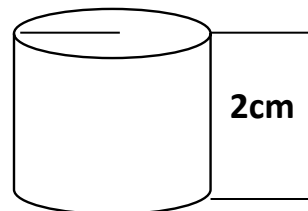


Corrective factor
via MonteCarlo simulation



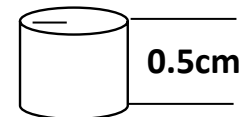
$$E_{Sample/STD} = \frac{Efficiency_{Sample}}{Efficiency_{STD}}$$

Acrylic Sample

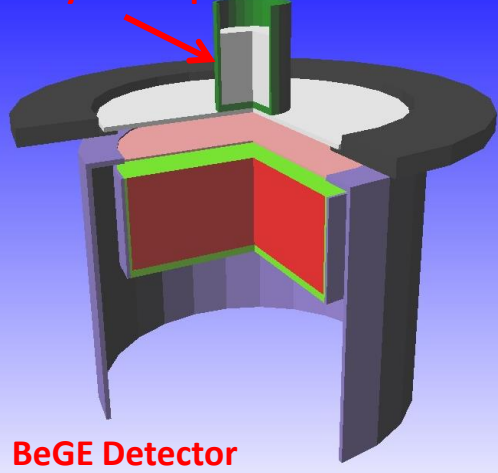


STD liquid

0.3cm

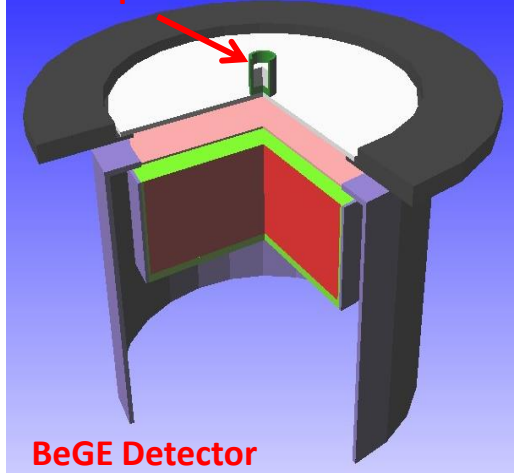


Acrylic Sample



BeGe Detector

STD liquid



BeGe Detector

	GeGEM	GeKan	BeGE
$E_{Sample/STD}$	1.55	1.3	1.31

$$C_{Sample} \left(\frac{g}{g} \right) = I.A. \cdot \frac{Counts_{Sample}}{Counts_{STD}} \cdot \frac{C_{STD} M_{STD}}{M_{Sample}} \cdot \frac{e^{-\lambda T_{Wait-STD}}}{e^{-\lambda T_{Wait-Sample}}} \cdot \frac{(1 - e^{-\lambda T_{Mis-STD}})}{(1 - e^{-\lambda T_{Mis-Sample}})} \cdot E_{Sample/STD}$$

NAA: preparation of irradiation (December 2015)

Samples were prepared together with the NAA standards for neutron irradiation in the **Central Thimble** (Flux of neutrons: $10^{13} \text{ cm}^{-2} \text{ s}^{-1}$)



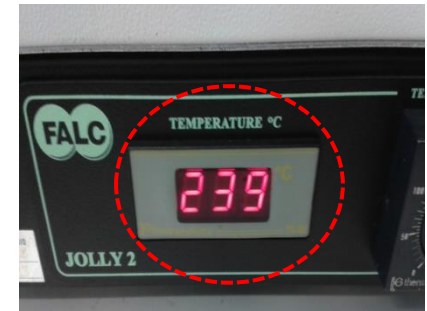
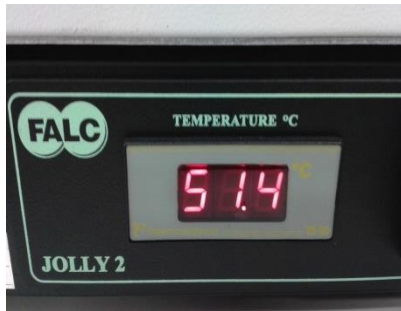
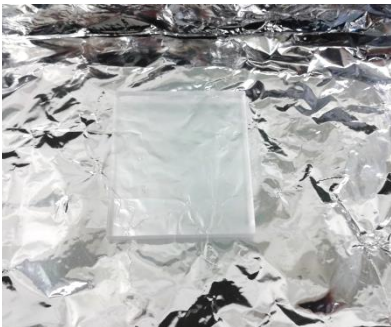
Preparation of the standards inside quartz vials



Acrylic sample + standards in the irradiation containers (one per sample)



The Acrylic cylinders were destroyed!



Not a question of temperature inside the reactor core ($\sim 70\text{-}80^\circ \text{C}$)